【2019年度】

国際会議 海外発表

日付	講演・会議・掲載誌名など	テーマ
2019年8月 (オンライン公開)	日本原子力学会 英文論文誌	Development of the residual sodium quantificationmethod for a fuel assembly of SFRs
2019年7月14日~19日	ASME Pressure Vessels& Piping Conference 2019	Study on The Predictive Evaluation Method of Nonlinear Sloshing Wave Crest Impact Load on The Roof of Cylindrial Tanks
		Research and Development of Threedimensional Isolation System for sodium- Cooled Fast Reactor: Part 3: Characteristics for Elements on Basis of Half-Scale Loading Tests
		Research and Development of Threedimensional Isolation System for sodium- Cooled Fast Reactor: Part 4:Proposal of Optimal Combination Method to Minimize Variation for Vertical Stiffnes
		Fast Reactor Core Seismic Analysis for Verification of Assessment Model Considering Deformation of Core Elements
		Eeffect of Deformation of Core Elements of Fast Reactor Core to The Seismic Response
2019年5月19日~24日	ICONE27 (The 27th International Conference on Nuclear Engineering)	Solidification behavior of melt materials contactingwith coolant
		Study on the application of machine learning todebris bed coolability evaluations for a sodium-cooled fast reactor
		Coolability evaluation of the debris bed on corecatcher in a sodium-cooled fast reactor
		Holding force tests of Curie Point Electro- Magnetin hot gas for passive shutdown system
		A Conceptual Design Study on Pool-type Sodium- cooled Fast Reactor with Enhanced Anti- seismicCapability
		In-Vessel Thermal-Hydraulics Analyses of The ASTRID-600MWe Reactor with STAR-CCM+ Code to Supply Boundary Conditions for Mechanical Evaluation
2019年5月12日~15日	ICAPP 2019 (2019 International Congresson Advances in Nuclear Power Plants)	Routing Study of Above Core Structure with Mock-up Experiment for ASTRID
		Dry Cleaning Process Test for Fuel Assembly of Fast Reactor Plant System (I) Pilot Scale Test forFuel Pin Bundle
		Dry Cleaning Process Test for Fuel Assembly of Fast Reactor Plant System (II) Laboratory ScaleTest for Fuel Assembly and Evaluation of theAmount of Residual Sodium
		Development of Under Sodium Viewer for NextGeneration Sodium-cooled Fast Reactors - Imaging Test in Sodium-

【2019年度】

国際会議・海外発表

日付	講演・会議・掲載誌名など	テーマ
	(2019 International Congresson Advances in Nuclear	Comparison of Sodium Fast Reactor CoreAssembly Seismic Evaluation using the JapaneseJAEA/MFBR/MHI and French CEA simulation tools