

【2017年度】

国際会議・海外発表

日付	講演・会議・掲載誌名など	テーマ
2018年3月28日	the 7th Joint IAEA-GIF Technical Meeting/ Workshop on the Safety of Liquid Metal Cooled Fast Reactors	Design evolution of Japanese Generation-IV SFR based on the GIF SDC and SDG
2017年7月 (オンライン公開)	Physical Review E	Lattice Boltzmann modeling and simulation of liquid jet breakup
2017年7月16日～20日	ASME 2017 Pressure Vessels & Piping Conference	Research and Development of Three Dimensional Seismic Isolation System Utilized Coned-Disc-Springs with Rubber Bearings
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor: (Part5) Non-Linear Analytical Models for Thick Rubber Bearing
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor: (Part6) Proposal of New Type of Hysteresis Model for Ultimate Behavior
		Fundamental study on the evaluation method of nonlinear sloshing wave height of the cylindrical tanks
		Core Seismic Experiment and Analysis of a Large Number of Element Models for Fast Reactor
		Core Seismic Experiment and Analysis of Full Scale Single Model for Fast Reactor
		Core Seismic Experiment and Analysis of Hexagonal Bundle Model for Fast Reactor
2017年7月2日～6日	ICONE25	CFD analysis technique development for wire contact effect of fuel rods within next generation FBR in Japan
		Reinforcement of Emergency Power Supply System for Next Generation SFR
		Lattice Boltzmann Simulation of Jet Breakup and Droplet Formation in Immiscible Liquid-Liquid System
2017年6月26日～29日	FR17 (International Conference on Fast Reactors and Related Fuel Cycles:Next Generation Nuclear Systems for Sustainable Development)	Feasibility of MA Transmutation by (MA, Zr)Hx in Radial Blanket Region of Fast Reactor and Plan of Technology Development
		SDC/SDG及びR&D(USAM、炉心耐震)の概要紹介【ポスター展示】
		Development of under sodium viewer for next generation sodium-cooled fast reactor
		Astrid Project, from Conceptual to Basic Design: Progress status
		Advanced sodium-cooled fast reactor development regarding GIF safety design criteria
		Study on Safety Design Concept for Future Sodium-Cooled Fast Reactors in Japan
2017年6月 (オンライン公開)	Mechanical Engineering Journal	Load and resistance factor design approach for seismic buckling of vessels

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2017年4月24日～28日	ICAPP2017 (2017 International Congress on Advances in Nuclear Power Plants)	Safety Evaluation of Self Actuated Shutdown System for Gen-IV SFR
		Discrepancy of Fast Reactor Neutron Flux Calculations Caused by Assembly Homogenization
		Model V&V and UQ procedure for a neutronics design methodology for next generation fast reactor
		Core performance requirements and design conditions for a next-generation sodium-cooled fast reactor in Japan
		Core design of the next-generation sodium-cooled fast reactor in Japan
		SAS4A analysis study on the initiation phase of ATWS events for Generation-IV loop-type SFR
		Japanese-French Collaboration on the Severe Accident Studies for ASTRID : Outcomes and future work program
		Dry Cleaning Process Test for Fuel Assembly of Fast Reactor Plant System
		Design study on measures to prevent loss of decay heat removal in a next generation sodium-cooled fast reactor
		The Collaboration of Japan and France on the Design of ASTRID Sodium Fast Reactor
		Status of ASTRID Sodium Fast Reactor Project:From Conceptual Design To Basic Design Phase
		Metallurgical investigations on creep rupture mechanisms of dissimilar welded joints between Gr.91 and 304SS
		Enhancement of Structural Design Margin for The Commercial Scale JSFR