

## 【2016年度】

## 国際会議・海外発表

日付	講演・会議・掲載誌名など	テーマ
2016年11月27日～30日	NTHAS10 (10th Japan-Korea Symposium on Nuclear Thermal Hydraulics and Safety)	Development of Core Hot Spot Evaluation Method of a Loop Type Fast Reactor Equipped with Natural Circulation Decay Heat Removal System
		Enhancement of Numerical Stability of Lattice Boltzmann Model for Immiscible Two-Phase Flow
2016年7月17日～21日	2016 ASME Pressure Vessels & Piping Conference	Development of Seismic Isolation systems for Sodium-Cooled Fast Reactor in Japan
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor:Part3.Ultimate Properties of a Half Scale Thick Rubber Bearings Based on Breaking Test
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor:Part4.Aging Properties of a Half Scale Thick Rubber Bearings Based on Breaking Test
		Core Seismic Experiment of a Full-Scale Single Model for a Fast Reactor
2016年6月	Nuclear Engineering and Design	Determination of In-service Inspection Requirements for Fast Reactor Components Using System Based Code Concept
2016年6月26日～30日	ICONE24 (International Conference on Nuclear Engineering)	Jet Breakup and Droplet Formation in Immiscible Liquid-Liquid System
		A Study on the Straight Double-Walled Tube Steam Generator Design against Sodium-Water Reaction in DECS
		Development of the Pump-Integrated Intermediate Heat Exchanger in Advanced Loop-Type Sodium-Cooled Fast Reactor for Demonstration
		Implementation of Reliability Evaluation into JSME Fast Reactor Codes (2) Development of a Code Case for Seismic Buckling Evaluation Based on LRFD
2016年4月17日～20日	ICAPP 2016 (International Congress on Advances in Nuclear Power Plants)	Flow-Induced Vibration of Primary Hot-Leg Piping in Advanced Loop-Type Sodium-Cooled Fast Reactor for Demonstration
		Study on reduction of vertical seismic response for Sodium-cooled Fast Reactor Building
		ASTRID Nuclear Island design: advances in French-Japanese joint team development of Decay Heat Removal systems