

## 【2015年度】

## 国際会議・海外発表 講演／発表

日付	講演・会議名	テーマ
2015年9月20日～24日	GLOBAL 2015	Enhancing MA Transmutation by Irradiation of (MA,Zr)Hx in FBR Blanket Region
2015年8月10日～14日	SMiRT 23 (Structural Mechanics in Reactor Technology)	Determination of ISI Requirements on the basis of system based code concept
2015年7月19日～23日	ASME 2015 Pressure Vessels & Piping Conference	Parametric Design Study about Seismic Isolation System for Fast Reactor JSFR
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor (Part1) Examination Plan
		Development on Rubber Bearings for Sodium-Cooled Fast Reactor (Part2) Fundamental Characteristics of Half-Scale Rubber Bearings Based on Static Test
		Evaluation Method of creep-fatigue life for 316FR weldment
		Fatigue Failure Evaluation Method and Fragility Curve Using Energy
2015年5月17日～21日	ICONE23 (International Conference on Nuclear Engineering)	Simplified Model for Highest Cladding Temperature Evaluation in Wire-wrapped Fuel Rod Bundle under Natural Circulation Decay Heat Removal
		Water Experiments on Thermal Striping in Reactor Vessel of Japan Sodium-cooled Fast Reactor - Countermeasures for Significant Temperature Fluctuation Generation-
		JSFR design progress related to development of safety design criteria for generation IV sodium-cooled fast reactors (1) Overview
		JSFR design progress related to development of safety design criteria for generation IV sodium-cooled fast reactors (2)Progress of safety design
		JSFR design progress related to development of safety design criteria for generation IV sodium-cooled fast reactors (3) Progress of component design
		SFR design progress related to development of safety design criteria for generation IV sodium-cooled fast reactors (4) Balance of Plant
		Development of JSME codes for fast reactors
		Numerical study on influence of Ohnesorge number and Reynolds number on the jet breakup behavior using the lattice Boltzmann method
		Experimental study on jet instability and breakup behavior in liquid-liquid system
2015年5月3日～6日	ICAPP 2015 (International Congress on Advances in Nuclear Power Plants)	Design study for reactor system of fast reactor JSFR -Concept of reactor system-
		Japan-France collaboration on The ASTRID Program and Sodium Fast Reactor

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国際会議・海外発表      雑誌等投稿

日付	雑誌・文献名	テーマ
2015年11月	Nuclear Technology	Severe External Hazard on Hypothetical JSFR in 2011
2015年5月	ICAPP 2015 (International Congress on Advances in Nuclear Power Plants)	Safety improvement in building layout design to meet the safety design criteria for the Generation IV SFR
		Study on a suppression of sodium-water reaction in SFR by applying sodium with suspended nanoparticles
2015年4月	Nuclear Engineering and Design	Reduced scale water test of natural circulation for decay heat removal in loop type sodium-cooled fast reactor