

## 【2011年度】 国際会議・海外発表

## 講演／発表

発表日	発表媒体	題名
2012年2月5日	ASME Code Week in Houston	Benefits from the SBC in New Reactor Design
2011年12月19日～20日	The 2nd workshop on Hydride Utilization in Nuclear Reactors	Core Characteristics of Large Fast Breeder Reactor with Hafnium Hydride Control Rods
		Hydrogen release and Firing experiments of Hafnium Hydride pellet
2011年12月11日～16日	GLOBAL2011	Technology Readiness Levels for Partitioning and Transmutation of Minor Actinides in Japan
		Technology Readiness of Partitioning and Transmutation toward Closed Fuel Cycle in Japan
2011年11月11日～16日	21st International Conference on Structural Mechanics in Reactor Technology (SMiRT 21)	Reasonableness Study for a Thermal Transient Load Modeling Method of Fast Breeder Reactor
		Experimental and Feasibility Study on Steel-Plate-Reinforced-Concrete Containment Vessel for Japan Sodium-Cooled Fast Reactor
2011年9月25日～29日	The 14th International Topical Meeting On Nuclear Reactor Thermalhydraulics (NURETH-14)	Development of Numerical Analysis Methods for Natural Circulation Decay Heat Removal System Applied to a Large Scale JSFR
		Sodium Experiment on Fully Natural Circulation Systems for Decay Heat Removal in Japan Sodium-Cooled Fast Reactor
		Water Test of Natural Circulation for Decay Heat Removal in JSFR
		Development of Core Hot Spot Evaluation Method for Natural Circulation Decay Heat Removal in Sodium Cooled Fast Reactor
2011年7月17日～21日	ASME 2011 Pressure Vessels and Piping Conference	Development of LBB Assessment Method for Japan Sodium Cooled Fast Reactor (JSFR) pipes (4) -Verification of Crack Opening Displacement Assessment Method for Thin Wall Pipes Made of Modified 9Cr-1Mo Steel-
		Development of LBB Assessment Method for Japan Sodium Cooled Fast Reactor (JSFR) pipes (5) -Crack Growth Assessment Method for Pipes Made of Modified 9Cr-1Mo Steel-
2011年5月2日～5日	International Congress on Advances in Nuclear Power Plants (ICAPP'2011)	Conceptual Design for a Large-Scale Japan Sodium-Cooled Fast Reactor (3) Core Design in JSFR

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## 雑誌等投稿

発表日	発表媒体	題名
2011年5月6日～19日	19th International Conference On Nuclear Engineering (ICONE 19)  *本会は震災により延期となりましたが、投稿とみなされた論文内容について掲載しております。	Water Experiments on Thermal Striping in Reactor Vessel of Japan Sodium-Cooled Fast Reactor(Countermeasures for Control Rods and Radial Blanket Assemblies)
		Development of PIRT for Fast Reactor under Natural Circulation Decay Heat Removal Operations
		Conceptual Design Study for the Demonstration Reactor of JSFR:(2) Core Design
		Conceptual Design Study for the Demonstration Reactor of JSFR:(3) Safety Design and Evaluation
		Conceptual Design Study for the Demonstration Reactor of JSFR:(4) Structural Design of Reactor Vessel
		Conceptual Design Study for the Demonstration Reactor of JSFR:(5) Reactor Cooling System Design
		Conceptual Design Study for the Demonstration Reactor of JSFR:(6) Fuel Handling System Design
		Preliminary Evaluation of JSFR Achievement Level to Risk Targets
		Study on In-Service Inspection and Repair Program and related plant design for Japan Sodium-cooled Fast Reactor (JSFR)
		Experimental investigations of steel plate reinforced concrete bearing wall for fast reactor containment vessel
		Development on Thermal Transient Load Modeling Method of Fast Breeder Reactor
		Study on Flow Induced Vibration Evaluation of Large Diameter Piping with Single Elbow for a Large Scale JSFR:(1) Current Status of Flow Induced Vibration Evaluation for JSFR Piping
		Study on Flow Induced Vibration Evaluation of Large Diameter Piping with Single Elbow for a Large Scale JSFR:(2) Pressure Fluctuation Characteristics and Vibration Analysis in 1/3 Scale Hot-Leg Piping Experiments under Swirl Inflow Conditions
		Study on Flow Induced Vibration Evaluation of Large Diameter Piping with Single Elbow for a Large Scale JSFR:(3) Pressure Fluctuation Characteristics in 1/3 Scale Hot-Leg Piping Experiments under Deflected Inflow Conditions Due to UIS Structures